

National Analytical Management Program (NAMP) U.S. Department of Energy Carlsbad Field Office

Radiochemistry Webinars Nuclear Fuel Cycle Series Chemistry and Radiochemistry of the Reactor Coolant System







In Cooperation with our University Partners





















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Meet the Presenter...

Dr. Robert Litman

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na Spectrometry Library Robert Litman, Ph.D., has been a researcher and practitioner of nuclear and radiochemical analysis for the past 42 years. He is well respected in the nuclear power industry as a specialist in ray energies radiochemistry, radiochemical instrumentation and plant systems ice for each corrosion. He has co-authored two chapters of MARLAP, and is currently one of a team of EMS consultants developing radiological laboratory guidance on radionuclide sample analyses in various matrices, radioactive sample screening, method validation, core radioanalytical laboratory operations, contamination, and rapid

radioanalytical methods. He authored the Primary Water Chemistry Guidelines on Radionuclides section of the EPRI PWR, and has been a significant contributor to the EPRI Primary-to-Secondary Leak Detection Guidelines. Dr. Litman has worked with the NRC in support of resolving GSI-191 issues (chemical effects following a loss of coolant accident) at current nuclear power plants and reviewed designs for addressing that safety issue for new nuclear power plants. His areas of technical expertise are gamma spectroscopy and radiochemical separations. Dr. Litman has been teaching courses in Radiochemistry and related special areas for the past 28 years.

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Chemistry and Radiochemistry of the Reactor Coolant System

Robert Litman, PhD



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TRAINING AND EDUCATION SUBCOMMITTEE



Objectives

- Review the basic operation of a PWR and why chemistry control is important
- Describe the chemistry control regime in a PWR
- Describe the effects of chemistry control on radionuclide activity in the RCS
- Identify the principal radionuclides in the RCS and how they are formed
- Identify changes to RCS radiochemistry as a function of fuel condition and plant operation

Typical Reactor Coolant System of a PWR



Diagram from http://www.nrc.gov/reactors/power.html



Diagram from http://westinghousenuclear.com/Operating-Plants/PWR/Products-and-Services/cid/54/Steam-Generators

A PWR Fuel Assembly



Courtesy of Westinghouse at

 $http://intergoogle.westinghousenuclear.com/search?q=fuel+assembly \&site=default_collection \&client=westinghouse_frontend \&proxystylesheet=westinghouse_frontend \&output=xml_no_dtd$

What is the Job of the Plant Chemist at a PWR?

Minimize Corrosion

Why Minimize Corrosion?

- Minimizes activation to radioactive species
- Maximizes plant efficiency and power output
- Minimizes the probability of fuel defects occurring
- Minimizes the release of radionuclides to the environment and dose to the public
- Minimizes contamination in the plant
- Minimizes dose to plant staff

RCS Principal Metallurgy

(% Surface Area Exposed to RCS)

- Stainless Steel (~5%)
 - Piping
 - Pumps
 - Vessel, SG bowl and pressurizer internal cladding
- Inconel 600, 690, X-750 or 718 (60-70%)
 - SG tubes
 - Upper and lower internals
- Zircalloy or Zirlo (20-30%)
 - Fuel clad
 - Grid straps



RCS Components

- Stainless Steel
 - -Accounts for ~5% of surface area in RCS
 - –Contains ~18% chromium, ~8% nickel, ~74% iron
 - -Has the highest general corrosion rate of all internal RCS surface materials (10⁻³ to 10⁻⁵ mil/yr)
 - -Forms two different layers of corrosion during initial power cycle

RCS Components

- Inconel
 - -Accounts for 60-70% of surface area
 - -Composition
 - ~74-77% nickel, 14-17% chromium, 6-10% iron
 - Minor constituents (0.01-1%): Al, Cu, Ti, Si, Mn, C, Co, and S

RCS Components

Zirconium alloys

- Zirc-4 (98% Zr, ~1.4% Sn, 0.2% Fe,~0.1% Cr)
- Zirlo (99%Zr, ~1%Nb, ~0.1% Sn, lower Fe and Cr than Zirc-4)
- Principally as the fuel cladding
- Accounts for about 20% of the surface area
- Also used as the grid straps on the fuel assemblies



What are the Contaminants of Concern in the RCS?

- Major:
 - -Chloride
 - -Fluoride
 - -Oxygen
 - -Sulfate
 - -Zeolite formers (Al, Ca, Mg, Na, Si)

• Minor

- -Nitrates
- -Ammonia
- -Organics

Contaminants

- Oxygen(<5 ppb)
- Anionic
 - Cl (<150 ppb)
 - F (<150 ppb)
 - SO₄²⁻ (<150 ppb)
- Scale formers
 - Ca (<40 ppb)
 - Mg (<40 ppb)
 - Al (<80 ppb)
 - Si (<100 ppb)

Principal Corrosion Mechanisms in the RCS

- General corrosion: all metals
- Intergranular attack
 - Primary water stress corrosion cracking (PWSCC): stainless, Inconel
 - –Hydriding: zirlo, zircalloy
- Crevice corrosion: stainless

RCS General Corrosion (stainless steel)

RCS Fluid Flow



RCS Corrosion Film Characteristics (stainless steel)

- Region 1 a *transient* solid layer
- Region 1 stability affected by:
 - -Reactor power (i.e., temperature)
 - -pH
 - -Hydraulic flow
 - -Other chemicals/contaminants in the RCS
- Refueling outage effects

RCS Metal Alloys

- Highly resistant to general corrosion
- Metal alloys are a mixture

The microstructural unit – the "grain"

Metal Grains

- No specific size or shape
- Grain boundary
- Intergranular area
- Annealing of the alloy



Stress

- Caused by the inexact alignment of the grain boundaries
 - -Somewhat relieved by annealing
 - Can be increased by the result of "cold work" of metals
- Present to some extent in all alloys
- *Eventually* causes stress corrosion cracking (SCC)

What Chemicals are Used in the RCS?

- Water
- Hydrogen
- Boric acid
- Lithium hydroxide
- Hydrazine
- Hydrogen peroxide

Oxygen

The effect of oxygen on ferrous materials: $Fe + O_2 \leftrightarrow FeO$ (1) No Water! $Fe + H_2O \leftrightarrow FeO + H_2$ (2) No Oxygen!

Because the RCS operates with water, (2) is the one we focus on

Hydrogen

Hydrogen is principally used to suppress the radiolytic cleavage of water by gamma radiation. Radiolysis produces oxygen and reactive radical species such as O•, •OH, •OOH, O₂•

$$2H_{2}O \leftrightarrow 2H_{2} + O_{2}$$
$$H_{2} + \bullet OH \leftrightarrow H \bullet + H_{2}O$$
etc.

Hydrogen

• Hydrogen maintains metals in *lower* oxidation states:

 $Ni_xFe_yO_z + H_2 \rightarrow Ni_xFe_yO_{z-1} + H_2O$

- Electrochemical potential
- Controls oxide form of Fe and Ni

Hydrogen

Too much hydrogen will hydride fuel cladding

$Zr^{0} + 2H_{2} \longrightarrow ZrH_{4}$

and also will destabilize CRUD layers

Effect of Hydrogen on CRUD



Boric Acid

- Soluble neutron absorber is ¹⁰B (19.9 %)
- Start of fuel cycle: 1200-1700 ppm B
- End of fuel cycle: o ppm B
- The "at temperature" pH of RCS is acidic without a pH control agent

Boric Acid Reactions

- Both isotopes of boron undergo nuclear reactions:
 - ${}^{10}B(n,\alpha)^7Li$
 - Cross section is ~3900 barns
 - ~0.15 ppm/day/1000 ppm boron
 - $-{}^{11}B(n,\gamma){}^{12}B \rightarrow {}^{12}C$
 - Cross-section is 5 mb

Control of RCS at pH_{Tavg} is "Critical"

- Boric acid makes the RCS acidic
- LiOH is added pH_{Tavg} to about 7.2
 Neutrality for pH_{Tavg} is about 6.9
- Ratio of B/Li maintained by Chemistry and Operations groups

Why Lithium?

- No long-lived activation products
- Easily enriched to the 7Li isotope
- Small amount in mass required to adjust pH
- It is produced by boron reaction

Lithium Concentration During Power Operation

- Lithium is also "used up" by:
 - RCS bleed and feed
 - To maintain power level by diluting out B
 - Chemistry sampling
 - Cation bed in service
 - RCP seal leak off
- Certain operational changes require addition of lithium

Lithium Concentration During Power Operation

Produced by

- ¹⁰B(n, α)⁷Li reaction
- Cross section is ~3900 barns
- $\sim 0.15 \text{ ppm/day/1000 ppm boron}$
- Used up by
 - ⁷Li(n,n α)³H
 - This is NOT a significant contributor to RCS ³H

$$-$$
 ⁷Li(n, γ)⁸Li \rightarrow ⁸Be + β ⁴

(⁸Li $t_{1/2}$ = 840 ms) (⁸Be $t_{1/2}$ = 7x10⁻¹⁷ s)

What About 6Li?

- Natural lithium is 7.6% ⁶Li
 - -PWR lithium is enriched to 99.5+% ⁷Li
 - -Minimizes production of tritium
 - ${}^{6}\text{Li}(n,\alpha){}^{3}\text{H}$

Lithium and Boron Effects

(Constant Temperature)



Hydrazine

- Added during start-up from refueling outages
- Reacts with oxygen $-N_2H_4 + O_2 \rightarrow N_2 + 2H_2O$
- Decomposes
 - $-N2H4 \rightarrow 2H2 + N2$
 - $-2N2H4 \rightarrow N2 + 2NH3 + H2$

Hydrogen Peroxide

- Transient corrosion layer
- Decomposes into H₂O and O₂
- Provides an oxidizing environment to solubilize ions

Activation Products: Corrosion and Contaminants

Radionuclide Product	Half-Life	Nuclear Reaction	Source of Target Material	Principal Gamma Ravs (keV)
⁵⁶ Mn	2.6 h	⁵⁵ Mn(n, γ)	Corrosion	847
²⁴ Na	15 h	²³ Na(n, γ)	MUW, SAT	1369, 2754
⁴¹ Ar	1.8 h	40 Ar(n, γ)	Air contaminant	1294
58Co	71 d	⁵⁸ Ni(n, p)	Nickel alloys	811
¹²² Sb	2.7 d	$^{121}\mathrm{Sb}(n,\gamma)$	Start up source, RCP bearing	564
⁸⁹ Zr	3.3 d	⁹⁰ Zr(n, 2n)	Fuel cladding	909
¹⁸⁷ W	24 h	¹⁸⁶ W(n, γ)	Weld rod residue, satellite	686
51 Cr	28 d	${}^{50}Cr(n,\gamma)$	Stainless steel, nickel alloys	320
⁵⁴ Mn	312 d	⁵⁴ Fe(n, p)	Stainless steel	835
⁶⁰ C0	5.3 a	⁵⁹ Co(n, γ)	Cobalt bearing components	1332, 1173
³⁸ Cl	37 m	³⁷ Cl(n, γ)	MUW, H_3BO_3 , contaminants	2168, 1642
¹²⁴ Sb	60 d	$^{123}{ m Sb}(n,\gamma)$	Start-up source, RCP bearing	603, 1691
¹²⁵ Sb	2.8 a	$^{124}\text{Sb}(n,\gamma)$	alloy, fuel clad	428, 601
^{110m} Ag	250 d	109 Ag(n, γ)	Control rod wear	658, 885
^{108m} Ag	420 a	¹⁰⁷ Ag(n, γ)		723, 434
⁹⁴ Nb	2 x 10 ⁴ a	93 Nb (n, γ),	Fuel clad, fuel	871, 703
		Fission Product		
⁴⁶ Sc	84 d	45 Sc(n, γ)	Trace element in zirc cladding	1120, 889
57 Co	272 d	⁵⁷ Fe(p, n)	Stainless steel	122, 137
¹⁰⁹ Cd	461 d	$^{108}{ m Cd}(n,\gamma)$	Control rod	88
^{115m} Cd	45 d	¹¹⁴ Cd(n, γ)		934, 1291

Radionuclides: Activation of Water and Chemicals

Radionuclide Product	Half Life	Nuclear Reaction	Source of Target Material	Principal Gamma Rays (keV)
⁷ Be	53 d	⁷ Li(p, n), Fission Product	Li	478
3 H	1 2.3 a	¹⁰ B(n,2α), Fission Product ⁶ Li(n,α)	H ₃ BO ₃ , fuel, Li	None (β ⁻ emitter)
¹¹ C	20 m	¹¹ B(p, n)	Boric Acid	511 (See Note)
¹⁴ C	5715 a	$^{14}N(n, p), ^{17}O(n, \alpha)$	RCS, MUW	No gamma
¹³ N	10 m	¹⁶ O(p, α)	Reactor Coolant	511 (See Note)
¹⁶ N	7.1 S	¹⁶ O(n,p)	Reactor Coolant	6129, 7115
¹⁸ F	109 m	¹⁸ O(p,n)	Water	511 (See Note)
⁶⁵ Zn	244 d	64 Zn(n, γ)	Zinc acetate or oxide	1115
^{69m} Zn	14 h	68 Zn(n, γ)	Zinc acetate or oxide	439
^{71m} Zn	4 h	70 Zn(n, γ)	Zinc acetate or oxide	387

Note: This is positron annihilation radiation because ¹¹C, ¹³N and ¹⁸F decay by positron emission. The positron interacts with an electron, and two 511 keV gamma rays are emitted. There is no direct gamma ray associated with ¹¹C, ¹³N or ¹⁸F decay.

Fission Product Radionuclides

Short Half-life	t _{1/2}	Intermediate Half-life	t _{1/2}
^{85m} Kr	4.5 h	¹³¹ I	8.0 d
⁸⁷ Kr	1.3 h	¹³³ Xe	5.2 d
⁸⁸ Kr	2.8 h	⁹⁹ Mo	2.7 d
^{99m} Tc	6.0 h	¹⁰⁶ Ru	1.0 y
97 Nb	1.2 h	95Zr	64 d
^{135m} Xe	15 min	⁹⁵ Nb	35 d
¹³⁵ Xe	9.1 h	¹³⁷ Cs	30 y
¹³⁸ Xe	14.1 min	¹³⁴ Cs	2.1 y
97 Zr	17 h	¹³⁶ Cs	13.1 d
132]	2.3 h	^{133m} Xe	2.2 d
133]	21 h	¹⁴⁰ La	1.7 d
¹ 34	53 min	¹⁴⁰ Ba	12.8 d
135]	6.6 h	¹⁰³ Ru	39 d
⁸⁸ Rb	18 min	¹⁴⁷ Nd	11 d

Radionuclides Formed from the Activation of Fuel

Radionuclide	Half-Life of Radionuclide	Nuclear Reaction	Source of	Principal Gamma Rays
Troduct	Product	Kedection	Material	(keV)
²⁴¹ Am	433 a	MNC	²³⁸ U	59
²³⁹ Pu	2.4 x 10 ⁴ a	MNC	238U	None detectable
²⁴⁰ Pu	6.6 x 10 ³ a			at RCS
²⁴¹ Pu	14 a			concentrations
²⁴² Cm	163 d	MNC	²³⁸ U	None detectable
²⁴³ Cm	29.1 a			at RCS concentrations
² 39U	23 m	²³⁸ U(n, γ)	²³⁸ U	None detectable
				at RCS
				concentrations
²³⁹ Np	2.4 d	Decay of ²³⁹ U	²³⁸ U	106,278

Fuel Defect



Summary

- Corrosion in a PWR RCS is minimized to:
 - Maintain component integrity
 - Maintain good fuel performance
 - -Minimize the activation of corrosion products
- Control of contaminants and pH help to achieve these objectives

References

- EPRI, "Pressurized Water Reactor Primary Water Chemistry Guidelines" (Volume 1, Revision 6)
- Data Source for Half-lives and Cross Sections: <u>National</u> <u>Nuclear Data Center</u>, <u>Brookhaven National Laboratory</u>, based on <u>ENSDF</u> and the <u>Nuclear Wallet Cards</u>, <u>www.nndc.bnl.gov/chart</u>
- Diagrams from:
 - -<u>http://westinghousenuclear.com/Operating-</u> Plants/PWR/Products-and-Services/cid/54/Steam-<u>Generators</u>
 - <u>http://intergoogle.westinghousenuclear.com/search?q</u> =fuel+assembly&site=default_collection&client=westi nghouse_frontend&proxystylesheet=westinghouse_fr ontend&output=xml_no_dtd
 - -www.nrc.gov/reactors/power.html

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